Nanoscale Stable Precipitation-**Strengthened Steels for Nuclear Applications**

Nuclear Energy Enabling Technologies (NEET) Reactor Materials

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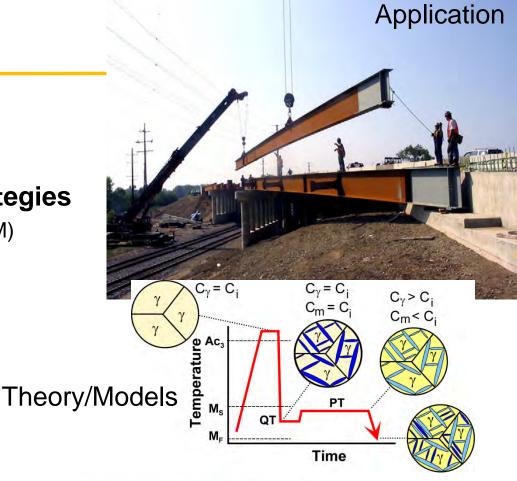






Overview

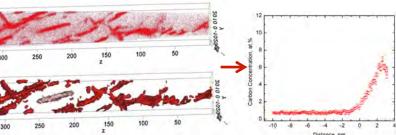
- Background
- Investigative Team
- Materials Processing Strategies
 - Colorado School of Mines (CSM)
 - Northwestern University (NU)
- Material production
- Experimental Plan
 - Thermal studies
 - Mechanical testing
 - Irradiation studies
 - Characterization







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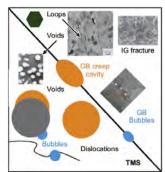
Project Background and Objectives

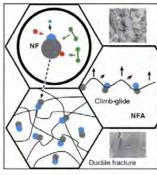
- Thermal conductivity
- Compatibility
- High-temperature strength
- Ductility
- Creep resistance
- Irradiation resistance
- Affordability

Nuclear Enabling Technologies (NEET) Workshop Report, July 29, 2010, Rockville, Maryland, accessed March 9, 2012 at http://www.ne.doe.gov/pdfFiles/ Neet Workshop 07292010.pdf



Stable, nanoscale precipitates and stable dislocation sinks improve irradiation resistance in steels.





Odette, G.R., Alinger, M.J., and Wirth, B.D., Annu. Rev. Mater. Res., vol. 38, 2008, pp. 471-503.

Create precipitation-hardened steels with improved thermal stability.

- Develop alloying strategies to improve irradiation performance of steels.
- Manufacture by conventional methods, with focus on manufacturability and affordability.
- Produce desired microstructures: stable precipitates and dislocation structures.
- Perform ion-beam experiments to determine irradiation resistance.
- Examine microstructural effects on mechanical properties.



Partnership - CSM, NU, NETL and LANL

NETL

- Produce and process steels with desired compositions and microstructures
- Thermodynamic/kinetic expertise for alloy design

CSM - Advanced Steel Processing and Products Research Center

- Global leader in automotive and structural steel development
- History of developing novel steels using thermal/mechanical processing

NU - Infrastructure Technology Institute

- Pioneers of tailored-precipitation hardened steels for structural uses
- Understanding of precipitation strengthening and kinetic control

McCormick

Northwestern Engineering

LANL

- Materials Science & Technology: Metallurgy (MST-6) steel physical metallurgy expertise
- Facilities for materials characterization
- Irradiation testing (IBML)





Advanced High-Cr martensitic or ferritic-martensitic steels

- Traditional material in reactors
- Optimize alloying and deformation processing for strength and creep resistance

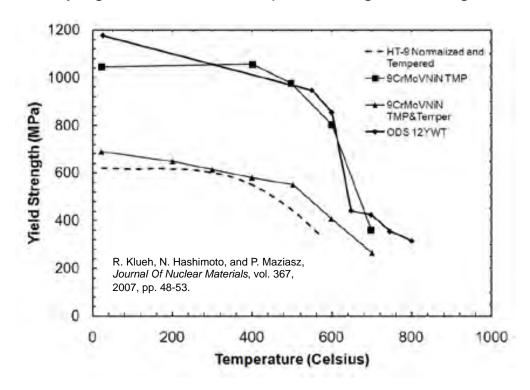


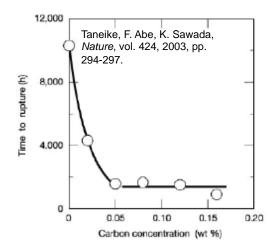


Table 1 - Composition of NF616, wt. pct.

C	Si	Mn	Cr	Mo	W	V	Nb	В	N
0.07	0.06	0.45	9.0	0.5	1.8	0.2	0.05	0.004	0.06

Base alloy, NF616 - Normalized and Tempered

- Lower C content to promote MX precipitation over M₂₃C₆
 - More resistant to coarsening
 - Excellent creep strength
- Increased Mn and N
 - Stabilize austenite for thermomechanical processing
 - Coarsening resistance (VN)
- Mo, W alloying optimized for solid solution strengthening



What is effect of C content on precipitation, aging stability, and resulting mechanical behavior?

To what extent can excess N improve austenite stability and coarsening resistance?

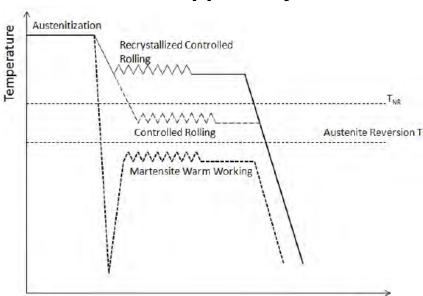




Thermomechanical Processing (TMP)

- Thermal solutionizing treatments
- Cooling rate
- Hot- or warm-working

Initial Material supplied by NETL





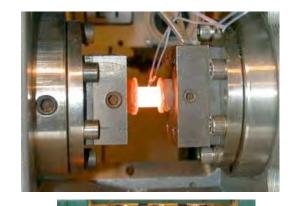


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Time



- Thermomechanical Processing (TMP)
 - Recrystallized controlled rolling
 - Decrease austenite grain size for nucleation sites
 - Decrease martensite packet/block size for strength
 - Increased interfacial area
 - Controlled rolling
 - Retain dislocation substructure for nucleation sites
 - Increased strength
 - Martensite warm working
 - Precipitate orientation relationship to matrix altered
 - Control precipitate growth kinetics
- Laboratory-scale processing for promising processing routes

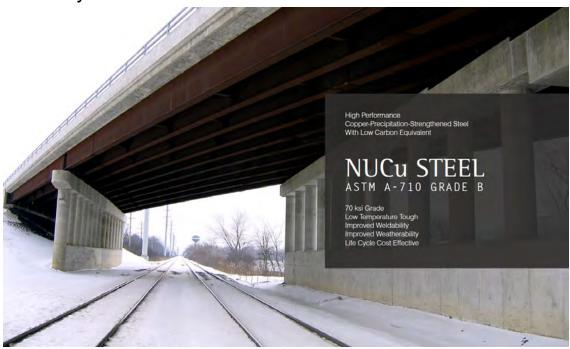






Novel Tailored-Precipitate Ferritic (TPF) steels

- Designed nanoscale fine Al, Cu, Nb, or Mo-based precipitates plus deformation processing.
- Optimize for strength, corrosion resistance, irradiation resistance, weldability, and thermal stability.



Front Cover:

Bridge in Lake Villa, Illinois, build with NUCu 70W (ASTM A710 Grade B) Steel





- Corrosion resistant structural steels tailored at NU
 - 1.3-2.5 Cu, 2.7Ni, 0.6Al
 - YS = 1000 Mpa
 - El% = > 20%
 - Impact energy > 70 J at -40 °C
 - Unique structure Ni-Al "shell"
 - Initial irradiation testing being initiated

- Te Cu Ni Al
- NU results suggest NiAl precipitates can be optimized for irradiation resistance (i.e., 17-7 PH stainless steels)
 - Meets strength and ductility requirements
 - Precipitate morphology shown to be adjustable with processing
 - High-temperature stability (700 °C)
 - Mo impedes morphology change/coalescence Mo migrates to interface



H.A. Calderon, M.E. Fine, J.R. Weertman, *Met. Trans.* Vol. 19, 1988, pp. 1135-1146.

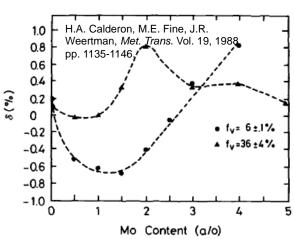
Nb or V carbide optimization





Design precipitates for application to nuclear reactors

- Alloys have been designed to take advantage of NiAl precipitate stability
 - NiAl stable ordered phase (B2 structure)
 - Wide composition band possible computational and experimental verification
- Quaternary alloying strategies, Mo
 - Control lattice parameter mismatch
 - Slow coarsening rate
 - Shape control
 - Size control
- Examination of Nb and V carbide behavior
- Thermomechanical processing



Can FeNiAl alloys be further optimized for use in irradiation-resistant materials?



Weldability

- Previous steels have been extremely weldable, which is rare in advanced, high-strength materials. Important for manufacturability.
- Controlled carbon equivalent and ability to control temperature during welding
 no brittle heat affected zone







Partnership - CSM, NU, NETL and LANL

Tailored-Precipitate Ferritic Steels (TPF)

- NiAl alloys have been produced at NETL
 - Four 5kg heats
 - 2.8Ni, 2.3Al
 - 2.6Ni, 3.0Al
 - 0.08C, 0.1Nb
 - 0.08C, 0.1V
- Aging studies underway at NU test thermal stability and mechanical properties as a function of time and temperature
- Irradiation studies being initiated at LANL

Advanced Hi-Cr ferritic or ferritic-martensitic steels

- NF616 produced at NETL
- Thermomechanical examinations underway at CSM
 - Irradiation studies being initiated at LANL









Experimental Plan - CSM, NU, NETL, and LANL

- Material Production and Thermomechanical processing
- Aging studies (CSM, NU)
 - 650 °C up to 5000 hours (>200 days)
 - Microstructural and mechanical evaluations to determine best-performing materials
- Mechanical Testing (CSM, NU, LANL)
 - Tensile testing to 700 °C
 - Creep-rupture testing
 - Nano-indentation
- Irradiation, Ion Beam Materials Laboratory (IBML) at LANL
 - Irradiation processes being determined
 - Mechanical and Microstructural response
- Microstructural Characterization and Interpretation (CSM, NU, LANL, NETL)
 - Precipitate analysis via TEM and EDS, along with 3DAP
 - EBSD for texture and grain-boundary type information
 - Understand relationships between processing, microstructure, and properties

Summary

- Develop stable nano-precipitates for reactor applications
- Use flexible processes and tailored compositions to produce desired microstructures
- Characterize microstructures with advanced techniques
- Optimize microstructures for reactor applications
- Initial iterations of materials designed and produced through conventional steelmaking processes
- First stages of testing underway



